

# Analysis of Gamma Ray Emissions From Fission Product Contributors to the Antineutrino Spectrum

- C. J. Martoff, Samuel Kim, Temple University
- Michael Dion, David Glasgow, Oak Ridge National Laboratory

## Reactor Antineutrino Anomaly (RAA)

- Name given to an apparent bump in the energy spectrum of antineutrinos from reactors, around 5 MeV energy

### Reactor Neutrino<sup>1</sup>

- From one fission, about 7 antineutrinos are produced.
- A 1 GW thermal reactor emits approximately 1E+20 antineutrinos per second
- A reactor produces over 1000 fission fragment nuclides, each of which beta decays.

### Modeling the Reactor Neutrino Spectrum

#### Dwyer and Langford<sup>2</sup>

- Ab-initio summation method with nuclear data from ENDF/B-VII.1
- RAA : spectral bump is shown at Antineutrino energy at 5 to 7 MeV (Positron energy at 4 to 6 MeV)
- Claim: resulting from strengths of eight beta decay branches in the tabulated nuclear data.
 

• 93-Rb (432.61 keV, 5.84 s)	100-Nb (535.67 keV, 1.5 s)	140-Cs (602.25 keV, 63.7 s)
• 95-Sr (685.6 keV, 23.9 s)	92-Rb (814.98 keV, 4.48 s)	96-Y (1750.4 keV, 5.34 s)
• 142-Cs (359.60 keV, 1.68 s)	97-Y (1103 keV, 1.7 s)	
- The tabulated Cumulative Fission Yields of these nuclides can be checked by their gamma ray emissions!

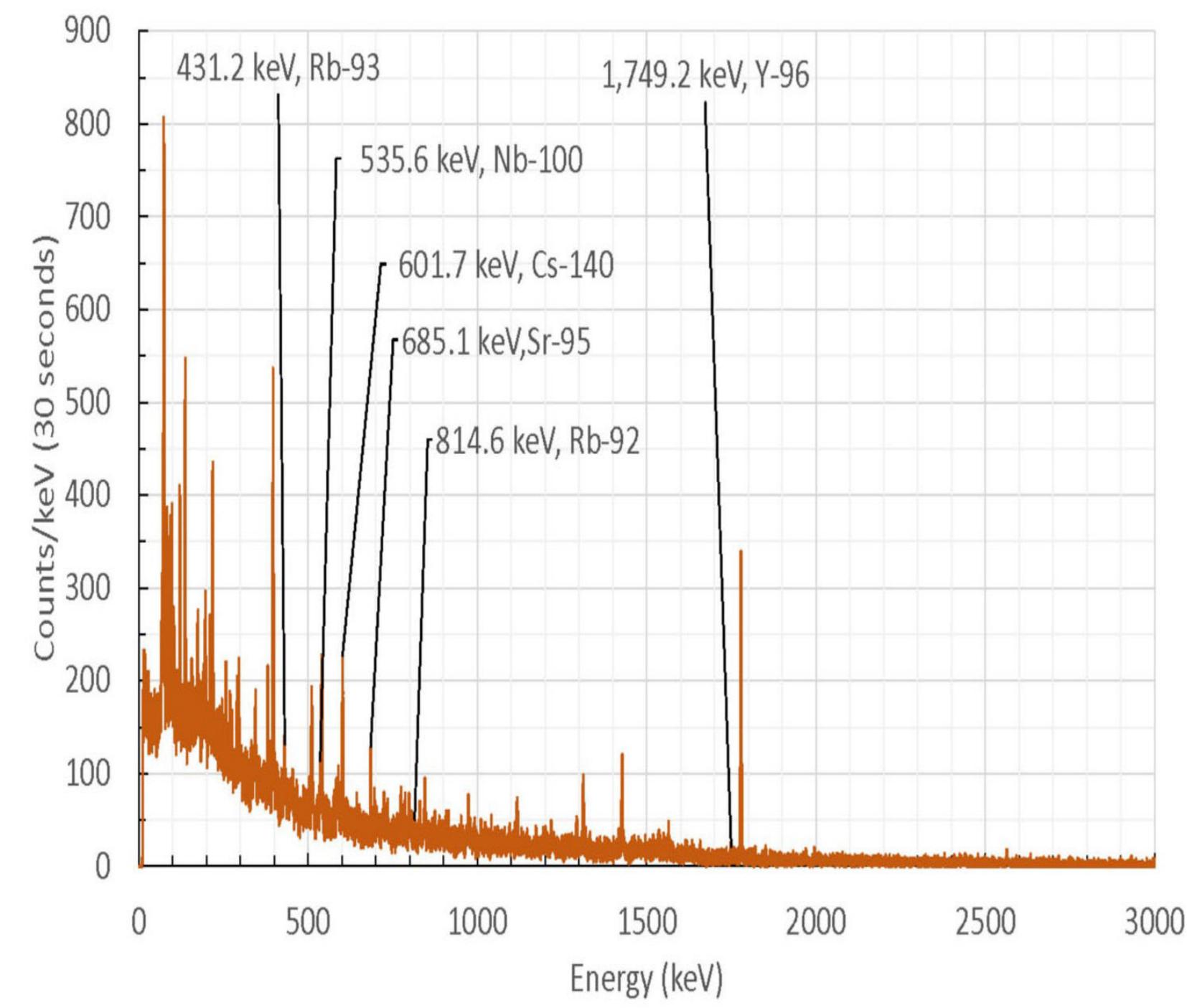
<sup>1</sup>Hayes and et al., 2012, *Reactor Antineutrino Flux & the Anomaly*, Applied Antineutrino Physics workshop

<sup>2</sup>Dwyer, Daniel A, and Thomas Langford, 2015, "Spectral Structure of Electron Antineutrinos from Nuclear Reactors", *Physical Review Letters* 114

## <sup>235</sup>U Sample Irradiation at ORNL HFIR NAA Facility\*

- Natural Uranium nitrate ICP calibration solution
- 252.72 nano gram
- Irradiate for 30 seconds using PT-2, HFIR\*, at NAA\*
- "Rabbit" transit time = 20 seconds
- 142-Cs and 97-Y have decayed away
  - Exploring alternate way to measure these
- Gamma ray emission is measured for 30 seconds using ORNL P-type high purity germanium detector

\*Oak Ridge National Laboratory (ORNL)  
\*High Flux Isotope Reactor (HFIR)  
\*Neutron Activation Analysis facility (NAA)



## Expected Gamma Ray Rates

### Expected net count

$$\lambda N f \varepsilon t_c \left( \frac{5.2E + 8}{10E + 10} \right)$$

### Uncertainty in expected net count<sup>2</sup>

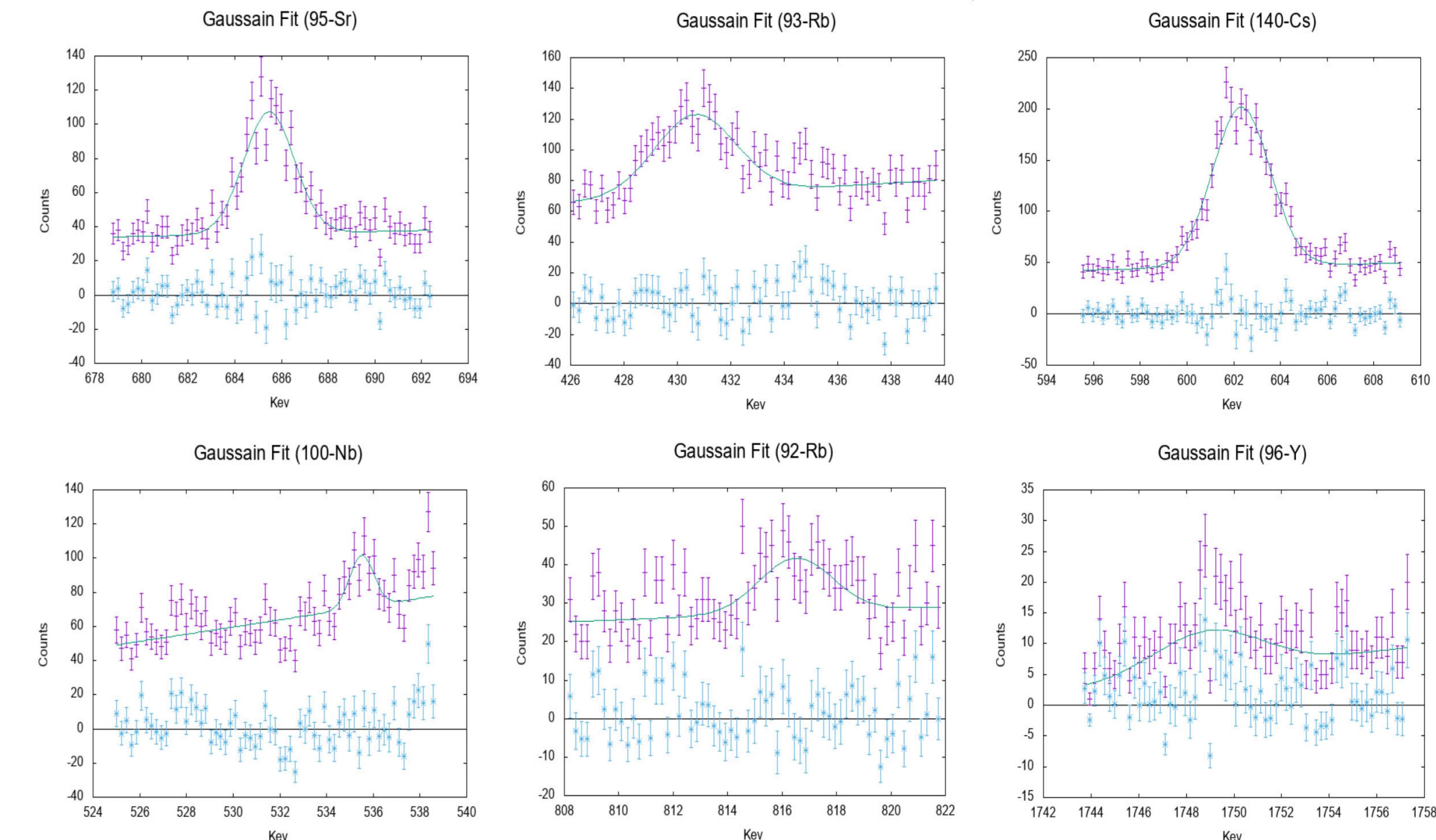
$$\frac{dc}{c} = \sqrt{\left(\frac{dN}{N}\right)^2 + \left(\frac{df}{f}\right)^2 + \left(\frac{d\lambda}{\lambda}\right)^2 + \left(\frac{de}{e}\right)^2}$$

<sup>1</sup>Robins, J, and et al, RadCalc: a program for estimating radiation intensity of radionuclide mixtures, *J Radioanal Nucl Chem* (2015) 303:1955–1960

<sup>2</sup>Taylor, John Robert. 1982. *An Introduction to Error Analysis: The Study of Uncertainties in Physical Measurements*. Mill Valley: University Science Books.

- Must solve Bateman equations for nuclides produced during irradiation (N)
  - Utilized RadCalc<sup>1</sup>
  - Outputs gamma rays / second emission rate from each product nuclide

- Efficiency ( $\varepsilon$ ) of ORNL HPGe – GEANT4 simulated
- Emission probability ( $f$ ) for selected gamma ray line
- Decay constant ( $\lambda$ )
- $C$  and  $dC$  are expected count and its uncertainty
- $N$  and  $dN$  are the number of nuclides fission produced and its uncertainty
- $f$  and  $df$  are the gamma ray emission probability and its uncertainty
- $\lambda$  and  $d\lambda$  are the decay constant and its uncertainty
- $\varepsilon$  and  $de$  are the GEANT4 simulated efficiency of ORNL HP detector and its uncertainty.



## Analysis of Observed Gamma Ray Peaks

### Nonlinear Peak Fitting (GNUPLOT)

#### Nonlinear fitting (gnuplot)

```
After 8 iterations the fit converged.
final sum of squares of residuals : 83.2073
rel. change during last iteration : 0
degrees of freedom (FIT_NDF) : 60
rms of residuals (FIT_STDFIT) = sqrt(WSSR/ndf) : 1.17762
variance of residuals (reduced chisquare) = WSSR/ndf : 1.38679
p-value of the Chisqr distribution (FIT_P) : 0.0253908

final set of parameters
=====
Asymptotic Standard Error
=====
amp1 = 479.356 +/- 18.06 (3.768%)
mu1 = 602.02 +/- 0.006 (0.007946%)
fwhm1 = 2.8909 +/- 0.1865 (3.213%)
a1 = 0.508822 +/- 0.001512 (51.34%)
b1 = -260.269 +/- 157.3 (60.43%)
correlation matrix of the fit parameters:
amp1 mu1 fwhm1 a1 b1
amp1 1.000 0.018 1.000 0.455 -0.207 1.000
mu1 0.018 1.000 0.455 -0.207 1.000
fwhm1 -0.022 -0.231 0.041 1.000
a1 0.018 0.231 -0.045 -1.000 1.000
b1
```

#### Fitted net count

- Gaussian (assumption) Asymptotic standard error
- Amplitude (amp1, 479)
  - mean (mu1, 602)
  - FWHM (fwhm1, 2.9)

#### Uncertainty

- Amplitude (amp1, +/- 18)
- Mean (mu1, +/- 0.05)
- FWHM (fwhm1, +/- 0.1)

## Alternate Analysis Method: Manual Peak Sum

Summation method: a peak profile is not assumed.

#### Net count

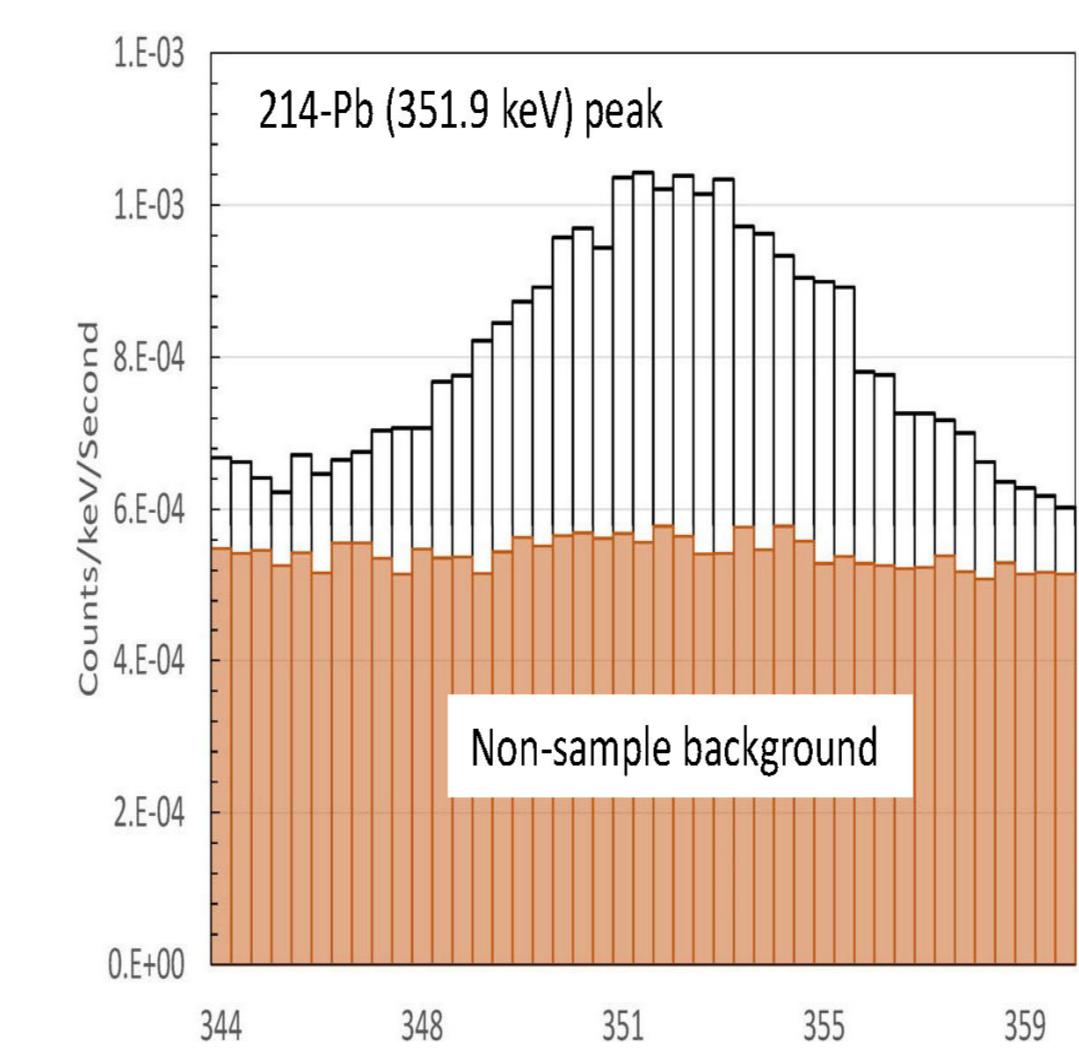
- Subtract gross non-sample peak count from sample peak
- Subtract continuum ( $\bar{B}'$ ) using trapezoid formula

$$N' = \sum_n (C' - D') - \frac{n}{2P} \left( \sum_{P_l} (C' - D') + \sum_{P_r} (C' - D') \right)$$

$$\sigma^2 = \bar{G}' + \sum_n \left( \frac{D'}{t_s} + \frac{D'}{t_b} \right)$$

Due to non-sample background

$$+ \left( \frac{1}{t_s} \right) \left( \frac{n}{2P} \right) \bar{B}' + \left( \frac{n}{2P} \right)^2 \left( \sum_{P_l} \left( \frac{D'}{t_s} + \frac{D'}{t_b} \right) + \sum_{P_r} \left( \frac{D'}{t_s} + \frac{D'}{t_b} \right) \right)$$



- Peak centroid: 352.8 keV
- FWHM: 2.1 keV
- Peak width search range: 3xFWHM using 3 channel averaged count
- Peak base width = 346.6 to 360 keV

## Predicted and Measured Fission Daughter Counts

### 95-Sr, 140-Cs, 92-Rb and 96-Y

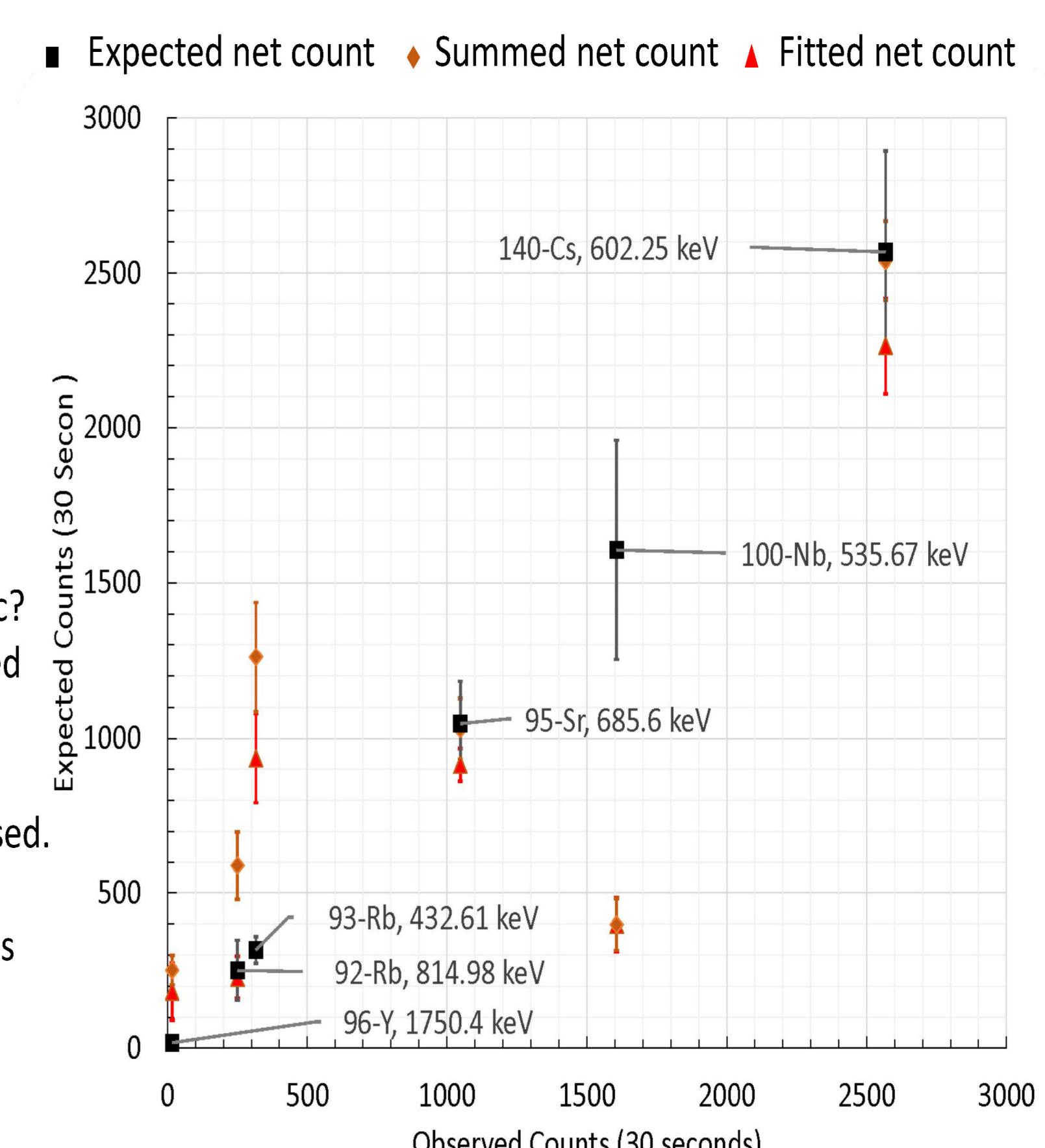
- Measured rates within  $2\sigma$  of expected rate

### 93-Rb (low) and 100-Nb (high)

- Measured rates are not within  $2\sigma$  expected rate
- Partial support for Dwyer & Langford proposed explanation of RAA

### Follow Up

- Errors in tabulated fission yields? Or in RadCalc?
- RadCalc uses ENDF\* VII, and Fitted and summed use ENDF VIII
- Better understanding about the systematic and random errors in RadCalc and analysis methods used.
- Refine the calculation models
- Further study is planned using more irradiations with larger samples at ORNL.



\*Evaluated Nuclear Data File (ENDF)